Docket No. 50-320

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Service List
(attached)

Mr. Gale K. Hovey
Vice President and
Director of TMI-2
Metropolitan Edison Company
P.O. Box 480
Middletown, Pennsylvania 17057

Dear Mr. Hovey:

The Nuclear Regulatory Commission has issued the enclosed Amendment No. 15 to License No. DPR-73. This amendment consists of changes to the Appendix B technical specifications and is in response to your requests dated February 23, 1981, and March 18, 1981. This amendment approves deletion of references to radioactive iodine, corrects the specified location for collecting tritium samples from the EPICOR-II ventilation system, deletes the requirement for analysis of short half-life gamma emitters in gaseous effluents from the EPICOR-II system, and deletes the requirement for annual Aerial Remote Sensing of cooling tower drift dispersions. The other changes requested in your February 23, 1981, letter will be addressed in a separate licensing action.

We have determined that the amendment involves an action which is insignificant from the standpoint of environmental impact and that there is reasonable assurance that the health and safety of the public will not be endangered by this action. Having made this determination, we have further concluded that pursuant to 10 CFR §51.5 (d) (4) an environmental impact statement, negative declaration or environmental impact appraisal need not be prepared in connection with the issuance of this amendment.

Copies of the related Safety Evaluation and the Notice of Issuance, which has been forwarded to the Office of the Federal Register for publication, are also enclosed.

Sincerely,

Bernard J. Snyder, Program Director
TMI Program Office
Office of Nuclear Reactor Regulation

Enclosures:
1. Amendment No. 15 to DPR-73
2. Safety Evaluation
3. Notice of Issuance

OFFICE		TMIPO:NRR	TMIPO:NRR	TMIPO:NRR	OELD	TMIPO:NRR
SURNAME	cc w/encl:	DBrinkman/pf	RWeller	OLynch	LChandler	BJSnyder
DATE	See next page	4/21/81	4/21/81	4/21/81	4/22/81	4/28/81

NRC FORM 318 (10-80) NRCM 0240 OFFICIAL RECORD COPY USGPO 1980—329-824

METROPOLITAN EDISON COMPANY

JERSEY CENTRAL POWER AND LIGHT COMPANY

PENNSYLVANIA ELECTRIC COMPANY

DOCKET NO. 50-320

THREE MILE ISLAND NUCLEAR STATION, UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 15
License No. DPR-73

1. The Nuclear Regulatory Commission (the Commission) has found that:

   A. The application for amendment by Metropolitan Edison Company, Jersey Central Power and Light Company, and Pennsylvania Electric Company (the Licensee) dated February 23, 1981, and supplemented on March 18, 1981, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the Order for Modification of License dated July 20, 1979, the Order of February 11, 1980, the Modification of Order dated August 11, 1980, the Amendment of Order dated November 14, 1980, the application for amendment, the provisions of the Act, and the rules and regulations of the Commission.

   C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and,

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, by changing paragraph 2.C.(2) to facility operating License No. DPR-73, to read as follows:

   2.C.(2) Technical Specifications

   The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 15, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications and all Commission Orders, issued subsequent to March 28, 1979.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Bernard J. Snyder

Bernard J. Snyder, Program Director
Three Mile Island Program Office
Office of Nuclear Reactor Regulation

Attachment:
Revised Technical Specifications

Date of Issuance: MAY 6 1981

FACILITY OPERATING LICENSE NO. DPR-73

DOCKET NO. 50-320

Replace the following pages of Appendix "B" Technical Specifications with the enclosed pages as indicated. The revised pages contain vertical lines indicating the area of change. The corresponding overleaf pages are also provided to maintain document completeness.

Pages

2-7
2-8
2-9
2-10d
2-11
2-12
2-13
2-14
2-14b
2-14c
3-21
3-22

LIMITING CONDITIONS FOR OPERATION	2.0 MONITORING REQUIREMENTS

2.1.2 Gaseous Effluents

Applicability

Applies to the controlled release of radioactive gases from TMI Unit Nos. 1 and 2.

Objective

To define the limits and conditions for the controlled release of radioactive gaseous effluents to the environs to ensure that these releases are as low as practicable. These releases should not result in radiation exposures in offsite areas greater than a few percent of background exposures. The instantaneous release rate for all effluent discharges should be within the limits specified in 10CFR 20.

To assure that the release of radioactive gases to offsite areas meet the as low as practicable concept, the following objectives apply:

a. The release rate of radionuclides, averaged over a yearly interval, except particulate nuclides with half lives greater than 8 days, discharged from Unit Nos. 1 and 2, should result in a dose rate at the site boundary of less than 5 mrem/yr to the whole body or any organ.

b. The release rate of particulate radionuclides with half-lives longer than 8 days, should result in a dose in the unrestricted area of less than 15 mrem/yr by inhalation or to the thyroid of a child through the cow-milk chain.

Specification

a. The instantaneous release rate of gross gaseous activity except for halogens and particulates with half-lives longer than eight days shall not exceed:

$$\sum \frac{Q_i}{(MPC)_i} \leq 1.5 \times 10^5 \frac{m^3}{sec}$$

Objective

To ensure that radioactive gaseous releases from the facility are within the limits of specifications.

Specification

During release of radioactive gaseous wastes, the following conditions shall be met:

A. During release of gaseous waste from the waste gas decay tanks, the following conditions shall be met:

LIMITING CONDITIONS FOR OPERATION 2.0 MONITORING REQUIREMENTS

Speeification (Cont'd)

f. Radioactive gas and particulates purged from the reactor building shall be filtered through the high efficiency particulate air filters.

g. The maximum activity to be contained in one gas decay tank shall not exceed 8800 curies (equivalent to Xe-133).

h. When the release rate of radioactive materials in gaseous wastes, averaged over a calendar quarter exceeds,

$$\sum \frac{Q_i}{(MPC)_i} \leq 6 \times 10^3 \frac{m^3}{sec}$$

(noble gas)

or

0.006 uCi/sec (particulates with half-lives greater than 8 days)

the licensee shall notify the NRC within 30 days, identifying the causes and describing the proposed program of action to reduce such release rates.

Specification (Cont'd)

3. The valves (Unit 1: AH-VIA and AH-VIB and Dampers; Unit 2: D5129 A/D and D5129 B/C) shall be interlocked to close or recirculate, respectively on receipt of a high radiation signal from the Reactor Building Exhaust Monitor (Unit 1: RM-A9; Unit 2: HP-R-225 and HP-R-226) respectively.

C. The flow rate for radioactive effluent streams and the Auxiliary and Fuel Handling Building and the Reactor Building, shall be monitored and recorded. Gaseous effluents from the Waste Gas Decay Tanks and the Reactor Building Purge Exhaust shall be continuoysly monitored and recorded.

D. Radioactive gaseous waste sampling and activity analysis shall be performed in accordance with Table 2.3-2.

E. The waste gas decay tank effluent monitor (Unit 1: RM-A7; Unit 2: WDG-R-1480) shall be tested using the installed check source or equivalent prior to any release of radioactive gas from a holdup tank and shall be calibrated quarterly using a referenced calibration source in a controlled reproducible geometry.

F. During power operation, the condensor vacuum pump discharge shall be continuously monitored for gross gaseous activity. The monitor shall not be inoperable for more than a week. Whenever this monitor is inoperable, a grab sample shall be taken daily and analyzed for gross radioactivity. (β, γ).

G. Facility records shall be maintained of radioactive concentration, release ratio and volume of each batch of gaseous effluents released.

TABLE 2.1-1 (Continued)

TABLE NOTATION

b. Tritium grab samples shall be taken at least once per 7 days from the ventilation exhaust from the Chemical Cleaning Building.

c. The principal gamma emitters for which the LLD specification applies exclusively are the following radionuclides: Mn-54, Fe-59, Co-58, Co-60, Zn-65, Mo-99, Cs-134, Cs-137, Ce-141 and Ce-144 for particulate emissions. This list does not mean that only these nuclides are to be detected and reported. Other peaks which are measurable and identifiable, together with the above nuclides, shall also be identified and reported. Nuclides which are below the LLD for the analyses shall be reported as "less than" the nuclide's LLD and shall not be reported as being present at the LLD level for that nuclide. The "less than" values shall not be used in the required dose calculations.

TABLE 2.3-2

Radioactive Gaseous Waste Sampling and Analysis (5)

Sample Type	Sampling Frequency	Type of Activity Analysis	Detectable Concentration (1)
Waste Gas Decay Tank Release			
Gas	Each Tank	H-3	10^{-6} uCi/cc
	Release	Individual Gamma Emitters	10^{-4} uCi/cc (2)
Reactor Building Purge Releases			
Gas	Each Purge	H-3	10^{-6} uCi/cc
		Individual Gamma Emitters	10^{-4} uCi/cc (2)
Condenser Vacuum Pump Releases			
Gas	Monthly	H-3	10^{-6} uCi/cc
	Monthly (3)	Individual Gamma Emitters	10^{-4} uCi/cc (2)
Auxiliary and Fuel Handling Building Exhaust Vent, Reactor Building Purge Vent and Unit Exhaust (6)			
Vent Release Points			
Gas	Monthly (4)	H-3	10^{-6} uCi/cc
		Individual Gamma Emitters	10^{-4} uCi/cc (2)
Charcoal	Weekly (7)	I-131, I-133, I-135	10^{-12} uCi/cc
Particulates	Weekly	Individual Gamma Emitters	10^{-10} uCi/cc (2)
	Monthly Composite	Sr-89, Sr-90	10^{-11} uCi/cc
	Monthly Composite	Gross Alpha Emitters	10^{-11} uCi/cc

(1) The above detectability limits are based on technical feasibility and on the potential significance in the environment of the quantities released. For some nuclides, lower detection limits may be readily achievable and when nuclides are measured below the stated limits, they should also be reported.

Bases (Cont'd)

These efforts should include consideration of meteorological conditions during releases.

The annual objectives have been developed taking into account a combination of system variables including fuel failures, primary system leakage, primary to secondary system leakage, and the performance of radionuclide removal mechanisms. I-131 is not specifically monitored because it has decayed to less than detectable activity since the March 28, 1979, accident.

Specification a. above, requires the licensee to limit the concentration of noble gases from the station to levels specified in 10 CFR 20, Appendix B, for unrestricted areas. Based on a X/Q of 6.7×10^{-6} sec/m^3, this specification provides assurance that no member of the general public would be exposed to radioactive materials in excess of limits specified in the Commission's rules and regulations.

Specification b. above, requires the licensee to limit the concentration of particulates with half-lives greater than eight days, released from the station to unrestricted areas to levels such that no individual will receive more than 500 mrem/yr to the total body or 3000 mrem/yr to the skin. The absence of iodine insures that the corresponding thyroid dose rate above background to an infant via the cow-milk-infant pathway is less than or equal to 1500 mrem/yr. A grazing period of 6 months has been applied to all radionuclides in particulate form with half-lives greater than eight days, to allow for the milk exposure pathway. The release rate is determined by using the methodology of Regulatory Guide I.109 (Rev. 1) and a relative deposition factor (D/Q) of 2.1×10^{-8} m^{-2}. The D/Q of 2.1×10^{-8} m^{-2} was calculated for the nearest cow located 1.2 miles SE of the station, using on-site meteorological data.

Bases (Cont'd)

5.6.1 of these Technical Specifications. On the basis of such reports and any additional information the Commission may obtain from the licensee or others, the Commission may from time to time require the licensee to take such action as the Commission deems appropriate.

TABLE 2.1-3a

RADIOACTIVE GASEOUS EFFLUENT MONITORING INSTRUMENTATION

INSTRUMENT	MINIMUM CHANNELS OPERABLE	APPLICABILITY	ACTION
10. EPICOR-II VENTILATION SYSTEM			
a. Noble Gas Activity Monitor	1	*	37
b. Deleted			
c. Particulate Sampler	1	*	41
d. Flow Rate Monitor	1	*	36
d. Sampler Flow Rate Monitor	1	*	36

TABLE NOTATION

*At all times.

ACTION 36 - With the number of channels OPERABLE less than required by the Minimum Channels OPERABLE requirement, effluent releases via this pathway may continue for up to 30 days provided the flow rate is estimated at least once per 4 hours.

ACTION 37 - With the number of channels OPERABLE less than required by the Minimum Channels OPERABLE requirement, effluent releases via this pathway may continue for up to 30 days provided grab samples are taken at least once per 8 hours and these samples are analyzed for gross activity within 24 hours.

ACTION 41 - With the number of channels OPERABLE less than required by the Minimum Channels OPERABLE requirement, effluent releases via the affected pathway may continue for up to 30 days provided samples are continuously collected with auxiliary sampling equipment as required in Table 2.1-1.

TABLE 2.1-3b

RADIOACTIVE GASEOUS EFFLUENT MONITORING
INSTRUMENTATION SURVEILLANCE REQUIREMENTS

	INSTRUMENT	CHANNEL CHECK	SOURCE CHECK	CHANNEL CALIBRATION	CHANNEL FUNCTIONAL TEST
10.	EPICOR-II VENTILATION SYSTEM				
	a. Noble Gas Activity Monitor	D	M	R(3)	Q(2)
	b. Deleted				
	c. Particulate Sampler	W	N.A.	N.A.	N.A.
	d. Flow Rate Monitor	D	N.A.	SA	SA
	e. Sampler Flow Rate Monitor	D	N.A.	SA	SA

TABLE NOTATION

(2) CHANNEL FUNCTIONAL TEST shall also demonstrate that control room alarm annunication occurs if any of the following conditions exist.

1. Instrument indicates measured levels above the alarm setpoint.
2. Circuit failure (alarm function only).
3. Instrument indicates a downside failure (alarm function only).
4. Instrument controls not set in operate mode or the switch position administratively monitored and controlled.

(3) The initial CHANNEL CALIBRATION shall be performed using one or more of the reference standards certified by the National Bureau of Standards or using standards that have been obtained from suppliers that participate in measurement assurance activities with NBS. These standards shall permit calibrating the system over its intended range of energy and measurement range. For subsequent CHANNEL CALIBRATION, sources that have been related to the initial calibration shall be used.

3.1.2.b Terrestrial
3.1.2.b(1) Aerial Remote Sensing

Environmental Monitoring Requirement

Deleted.

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

METROPOLITAN EDISON COMPANY

JERSEY CENTRAL POWER AND LIGHT COMPANY

PENNSYLVANIA ELECTRIC COMPANY

DOCKET NO. 50-320

THREE MILE ISLAND NUCLEAR STATION, UNIT NO. 2

Introduction

By letters dated February 23 and March 18, 1981, (reference 1, 2) the Metropolitan Edison Company (licensee) requested amendments to the Technical Specifications of Operating License No. DPR-73 for the Three Mile Island Nuclear Station, Unit 2. This evaluation discusses the requested amendments to Appendix B of Operating License No. DPR-73, namely: (1) Delete requirements to monitor for radioactive iodines in gaseous effluents; (2) Correct sampling locations for the Radioactive Gaseous Waste Sampling and Analysis Program (RGWSA); (3) Delete the Lower Limit of Detection (LLD) specification for gamma emitting gaseous emissions of the RGWSA; (4) Suspend the requirements for an Aerial Remote Monitoring Program to detect the environmental impact of salt drift from the cooling towers.

Evaluation

(1) Requirement to monitor for radioactive iodines in gaseous effluents.

Section 2.1.2 and 2.1.3 of the Environmental Technical Specifications (ETS) limit the gaseous effluents of I-131 and require the monitoring of radioactive iodines (I-131, I-133 and I-135) in gaseous effluents. I-131 is produced as a result of nuclear fission during reactor operation and these technical specifications ensure that the release of I-131 is within the regulatory limits.

Since the March 28, 1979 accident, the reactor has been shutdown and I-131 generation has ceased. With a decay constant of 0.086 per day, the I-131 concentration present at the time of the accident has decayed to less than detectable levels. Weekly monitoring of the charcoal sampler in the unit exhaust for I-133 and I-135 also has indicated less than detectable levels.

The staff has reviewed the licensee's request to delete the requirement to monitor radioactive iodines in the gaseous effluents. These factors have been considered: (a) Since the March 28, 1979 accident, I-131, I-133 and I-135 inventories have decayed, for approximately 80 half-lives in the case of I-131, to the extent that these radionuclides are no longer present in measurable quantities. For a total quantity of I-131, at the time of

reactor shutdown, of 6.6 x 10^7 Ci (reference 3), the present quantity of I-131 has decayed to less than 1 x 10^{-10} μCi; (b) throughout the cleanup and defueling operations, provisions will be taken to ensure that recriticality of the nuclear fuel will not occur. Detection of recriticality will primarily be based on direct monitoring of parameters (e.g., neutron flux, temperature, pressure) other than radioactive iodine generation; (c) although the analysis for radioactive iodines may not be required, the charcoal sampler will still be in place and analysis can be performed should it be warranted. Monitoring of gaseous and particulates with half-lives greater than 8 days, except for I-131, is still required.

Based on the above considerations for the fact that I-131 is not present, in measurable quantities and generation of I-131 will not occur throughout the cleanup and defueling operations, the staff has concluded that the deletion of requirements in Section 2.1.2 and 2.1.3 of the ETS in reference to I-131 gaseous effluents limits and radioactive monitoring as proposed by the licensee is appropriate. Therefore, deletion of those ETS requirements is approved.

(2) Correction of sampling location for the RGWSA.

Table 2.1-1 of the ETS requires a monthly sample of the EPICOR-II ventilation exhaust. Note b of Table 2.1-1 specifies the sample location to be the ventilation exhaust of the spent fuel pool area. The ventilation system associated with the EPICOR-II system, however, vents through the exhaust of the Chemical Cleaning Building and, therefore, the proposed change merely corrects the Table to properly reflect the actual location for the sample. Therefore, the staff approves the change as proposed by the licensee to amend Note b of Table 2.1-1 to specify the sample location to be the ventilation exhaust of the Chemical Cleaning Building.

(3) The LLD for principle gamma emitters for the RGWSA.

Table 2.1-1 of the ETS specifies that an analysis for principle gamma emitters be performed on the EPICOR-II ventilation of gaseous releases, and requires analyses for specific gaseous and particulate emissions. The radionuclides which are specifically required to be monitored for gaseous emissions are: Kr-87, Kr-88, Xe-133, Xe-133m, Xe-135 and Xe-138. The staff has evaluated the request to delete the monitoring of these radionuclides in gaseous effluents and has considered the following factors: (a) the longest half-life for these radionuclides is 5.27 days for Xe-133 and all these radionuclides have decayed to less than detectable levels. None of these isotopes are being generated as daughter products from other isotopes that are likely to be present in any significant quantity; (b) the specification to monitor principle gamma emitters for particulate emissions is still required; (c) the specifications to monitor gross gaseous activity and to sample and monitor for tritium effluents are still required.

Based upon the foregoing considerations, the staff has concluded that deletion of requirement in Table 2.1-1 of the ETS to analyze for specific gaseous emissions: Kr-87, Kr-88, Xe-133, Xe-133m and Xe-138, as proposed by the licensee, would be appropriate. Other airborne effluents from the EPICOR-II system that could be present in detectable quantities (i.e., particulates and tritium) as well as the gross gaseous activity would still be required to be monitored per Table 2.1-1 of the ETS. Therefore, the staff approves the deletion of the requirement in Table 2.1-1 of the ETS to specific analysis for principle gamma emitters in gaseous effluents.

(4) Requirements for an Aerial Remote Monitoring Program

Section 3.1.2.b(1) of the ETS requires an annual aerial photography program with correlating data from ground inspection surveys and drift modeling for purposes of interpretation and verification of the aerial photographs. The purpose of this surveillance program is to determine possible impacts to surrounding vegetative communities that may result from the operation of Unit 2 cooling towers. Such impacts may be associated with either episodic high-level or chronic low-level chlorine dosages due to cooling tower drift deposition. The ETS requirement also stipulates that this program shall be continued to two years at initial attainment of Unit 2 normal operation, and at that time, the licensee may request modification or termination of this monitoring requirement.

The staff has reviewed the monitoring program results for Unit 2 (NUREG-0738, "Investigation of Reported Plant and Animal Health Effects in the Three Mile Island Area") and has concluded that there has been no biological damage resulting from the routine operation of the Unit 2 cooling towers. In addition, the design use of the cooling towers has been suspended since the shutdown after the accident and any operations of the towers during the cleanup operations will be at greatly reduced levels.

Based on the above considerations, the staff concludes that the licensee's request to amend the ETS to suspend the aerial remote sensing and drift monitoring program should be approved since no ill-effects have been attributed to drift deposition from full operation levels and none will be likely to occur from low, intermittent operational levels.

Environmental Consideration

Based on the evaluation above, the proposed amendment to the ETS would not result in any environmental impact beyond those considered in the Final Programmatic Environmental Impact Statement, NUREG-0683 (reference 4) and the Final Supplement to the Final Environmental Statement for TMI Unit2, NUREG-0112 (reference 5). The staff has determined that this amendment does not authorize a change in effluent types or total amounts nor an increase in power level and will not result in any significant environmental impact. Having made this determination, the staff has further concluded that this amendment involves an action which is insignificant from the standpoint of environmental impact and, pursuant to 10 CFR 51.5(d) (4), that an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with the issuance of this amendment.

Conclusion

Based upon the staff's review of the proposed amendments to the Environmental Technical Specifications, the staff finds the licensee's request to be acceptable and grants the request. Based on the review, the staff has concluded that: (1) the modification does not authorize any significant change in the plant's operation, (2) the modification does not involve a significant increase in the probability or consequences of accidents previously considered or a significant reduction in a margin of safety and, therefore, does not involve a significant hazards consideration, (3) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the modified manner, and (4) such activities will be conducted in compliance with the Commission's regulations and the issuance of this modification will not be inimical to the common defense and security or to the health and safety of the public.

REFERENCES:

1. Technical Specification Change Request No. 26, Three Mile Island Nuclear Station, Unit 2, Docket No. 50-320, Metropolitan Edison Co., February 23, 1981.

2. Technical Specification Change Request No. 26, Addendum A, Three Mile Island Nuclear Station, Unit 2, Docket No. 50-320, Metropolitan Edison Co., March 18, 1981.

3. Table 11-1, Volume II, Part 2, "Three Mile Island, A Report to the Commissioners and to the Public," NRC Special Inquiry Group, January 1980.

4. Final Programmatic Environmental Impact Statement related to Decontamination and Disposal of Radioactive Wastes Resulting from March 28, 1979, Accident, Three Mile Island Nuclear Station, Unit 2, NUREG-0683, March 1981.

5. Final Supplement to the Final Environmental Statement, Three Mile Island Nuclear Station, Unit 2, NUREG-0112, December 1976.

UNITED STATES NUCLEAR REGULATORY COMMISSION

DOCKET NO. 50-320

METROPOLITAN EDISON COMPANY
JERSEY CENTRAL POWER AND LIGHT COMPANY
PENNSYLVANIA ELECTRIC COMPANY

NOTICE OF ISSUANCE OF AMENDMENT TO FACILITY OPERATING LICENSE

The U. S. Nuclear Regulatory Commission (the Commission) has issued Amendment No. 15 to Facility Operating License No. DPR-73, issued to Metropolitan Edison Company, Jersey Central Power and Light Company, and Pennsylvania Electric Company which changed Appendix B Technical Specifications for operation of the Three Mile Island Nuclear Station, Unit 2 (the facility) located in Dauphin County, Pennsylvania. The amendment is effective as of its date of issuance.

The amendment deletes references to radioactive iodine, corrects the specified location for collecting tritium samples from the EPICOR-II ventilation system, deletes the requirement for analysis of short half-life gamma emitters in gaseous effluents from the EPICOR-II system, and deletes the requirement for annual Aerial Remote Sensing of cooling tower drift dispersions.

The application for the amendment complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter 1, which are set forth in the license amendment. Prior public notice of this amendment was not required since the amendment does not involve a significant hazards consideration.

For further details with respect to this action, see (1) the application for amendment dated February 23, 1981, and amended March 18, 1981, (2) Amendment No. 15 to License No. DPR-73, and (3) the Commission's related Safety Evaluation. All of these items are available for public inspection at the Commission's Public Document Room, 1717 H Street, N.W., Washington, D.C. 20555 and at the Government Publications Section, State Library of Pennsylvania, Education Building, Commonwealth and Walnut Streets, Harrisburg, Pennsylvania 17126. A copy of items (2) and (3) may be obtained upon request addressed to the U.S. Nuclear Regulatory Commission, Washington, D.C. 20555, Attention: Director, TMI Program Office.

Dated at Bethesda, Maryland this 6th day of May, 1981

FOR THE NUCLEAR REGULATORY COMMISSION

Bernard J. Snyder, Program Director
TMI Program Office
Office of Nuclear Reactor Regulation